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Candu Reactor Severe Accident Analysis
2.1.1. CANDU ... For a pressurized heavy water reactor (PHWR), accidents that result in damage to the ... Severe accident analysis and accident management are addressed in a number of IAEA publications [1–9]. IAEA Safety Requirements publication on Safety of Nuclear Power Plants:

Analysis of Severe Accidents in Pressurized Heavy Water ...
Romania, as CANDU owner, is interested to have CANDU severe accidents analysis. ASTEC UE code should have a CANDU module in order to analyze PHWR nuclear reactors, besides the LWRs European reactors. The paper presents a simple model to analyze severe accident development, given the special features of the CANDU reactor, horizontal core, fuel channels in the pressure tubes, etc.

Severe accident development modeling and evaluation for CANDU
The CANDU reactor has moderator calandria vessel as ultimate heat sink during severe accident development, which acts also as a core debris catcher. 2. CANDU severe accidents analysis Nuclear reactor severe accidents are important in terms of consequences: radioactive releases and the public perception.

CANDU SEVERE ACCIDENT ANALYSIS - pub.ro
Preliminary Analysis of SB-LOCA-induced Severe Accident at CANDU-6 Reactor using M-CAISER Code Jun-young Kang a, Yong Mann Song a, Dong Gun Son a, Jong Yeob Jung a, Sang Ho Kim a, and Jun Ho Bae a* a Korea Atomic Energy Research Institute, Daejeon, 34057, Rep. of Korea *Corresponding author: kkang0620@kaeri.re.kr 1. Introduction

Preliminary Analysis of SB-LOCA-induced Severe Accident at ...
manageable size by using a scaling analysis based on geometric and stress-level similarity. In terms of analysis of these sequences, AECL and Ontario Power Generation Inc. are developing the MAAP-CANDU code to conduct severe core damage accident consequence analysis for CANDU reactors [6]. MAAP CANDU is the CANDU-version of the MAAP PWR code.

CANDU-Specific Severe Core Damage Accident Experiments in ...
The CANDU600 reactor uses natural uranium fuel and heavy water (D2O) as both moderator and coolant, with the moderator and coolant in separate systems. We chose to analyze accident development for a LOCA with simultaneous loss of moderator cooling and the loss of emergency core cooling system (ECCS).

A CANDU Severe Accident Analysis (Conference) | OSTI.GOV
Generic CANDU 6 plant severe accident analysis employing SCAPSIM/RELAP5 code

(PDF) Generic CANDU 6 plant severe accident analysis ...
Pg 3 Introduction • Presentation addresses Severe Core Damage Accident Analysis using MAAP4 CANDU • Severe Core Damage Accident – Accident in which substantial damage is done to the reactor core structure whether or not there are serious off-site consequences – Reactor Cooling System and Moderator back-up heat sinks are unavailable. In ACR-700, RWS must also fail (very unlikely scenario)

Severe Core Damage Accidents & MAAP4 CANDU.
Chapter 7 - Accident Analysis.wpd Rev. 8 November 9, 2009 22:26:44 vgs Chapter 7 - Accident Analysis Introduction This Chapter provides more specific information on performing accident analysis for CANDU. The process by which initiating events are selected is first discussed - some of this will review what we covered in Chapter 1.

Chapter 7 - Accident Analysis - nuceng.ca
24-May-01 CANDU Safety - #10 - Design and Analysis Process.ppt Rev. 0 15 Accident Classes EVENT CLASS (AECB C-6 Document) FREQUENCY RANGE (f) (per Reactor Year) ACCIDENT 1 10-2 ≤≤ f < 1 Off-Reactor Fuelling machine accident 2 10-3 ≤ f < 10-2 Single-channel events (FSB, EFF, PTR, FB) 3 10-4 ≤ f < 10-3 Large LOCA 4 10-5 ≤≤≤ f < 10-4 ...

CANDU Safety #10: Design and Analysis Process
pool formation and movement in case of a severe accident at a CANDU 6 reactor. Even if a severe accident never occurred in a CANDU reactor, the accident in Fukushima Daiichi, Japan illustrates why the research needs to be made to be able to understand the consequences of such an event. During a postulated severe accident in a CANDU power plant ...

3D NUMERICAL ANALYSIS OF MOLTEN DEBRIS DURING LATE PHASE ...
The progression of a severe core damage accident in a CANDU reactor is analyzed using the MAAP4 CANDU code, which is the CANDU-version of the MAAP code. AECL and Ontario Power Generation Inc., in co-operation with Fauske and Associates Inc., have developed the MAAP4 CANDU code for severe core damage accident analysis in a CANDU reactor.

Severe Core Damage Accident Progression within a CANDU ...
The paper overviews the analytical studies performed at Politehnica University of Bucharest on the analysis of late phase severe accident phenomena in a Canada Deuterium Uranium (CANDU) plant. The calculations start from a dry debris bed at the bottom of calandria vessel.

Analysis of Late Phase Severe Accident Phenomena in CANDU ...
Best-estimate severe accident and source term analysis for an ISLOCA scenario of a CANDU-6 plant using the MAAP-ISAAC code Article in Nuclear Engineering and Design 358:110443 · March 2020 with 3 ...

Best-estimate severe accident and source term analysis for ...
The MAAP-CANDU code is the primary code used for integrated severe accident analysis of CANDU reactor designs. It was developed from the MAAP-PWR code with special modules added to simulate the horizontal fuel channel and calandria vessel geometry of CANDU.

PRELIMINARY METHODOLOGY FOR THE ANALYSIS OF THE NATIONAL ...
Severe accident, CANDU, RELAP/SCDAPSIM, Best estimate, Uncertainty 1. INTRODUCTION One of the general attributes of a methodology to perform accident analysis of a nuclear power plant for the safety assessment is directly connected with the availability of qualified tools and analytical procedures suitable for this purpose.

BEPU ANALYSIS OF A STATION BLACKOUT IN CANDU REACTORS WITH ...
RSP-0246, An evaluation of severe accident computer codes for CANDU nuclear power plants RSP-0242, Verification of the PROMETHEUS core analysis code and its advanced pption RSP-0240, Technical basis for G-144 trip parameter acceptance criteria for the safety analysis of CANDU nuclear power plants

Safety analysis - Canadian Nuclear Safety Commission
Nor has any study on core meltdown accidents been done for the CANDU reactor (although initial examination of possible sequences is being sponsored as part of the AECB's research program). In the absence of relevant Canadian information, the work done by N. C. Rasmussen, as described in the Reactor Safety Study (WASH-1400) issued in 1975 by the U.S. Nuclear Regulatory Commission is used.

Findings on CANDU Reactor Accidents - CCNR
Keywords: CANDU, calandria vault, molten core–concrete interaction, failure, finite-element analysis 1 Introduction During a postulated severe accident in a generic CANDU 6 nuclear reactor, decay heat could cause the fuel channels to heat up and collapse to the bottom of the calandria vessel and then melt into a mixture called corium.

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